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## Original Article

# Reactor core analysis through the $SP_3$ -ACMFD approach. Part I: Static solution



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#### ABSTRACT

The present work proposes a solution to the static Boltzmann transport equation approximated by the simplified  $P_3$  ( $SP_3$ ) on angular, and the analytic coarse mesh finite difference (ACMFD) for spatial variables. Multi-group  $SP_3$ -ACMFD equations in 3D rectangular geometry are solved using the GMRES solution technique. As the core time dependent analysis necessitates the solution of an eigenvalue problem for an initial condition, this work is hence devoted to development and verification of the proposed static  $SP_3$ -ACMFD solver. A 3D multi-group static diffusion solver is also developed as a byproduct of this work to assess the improvement achieved using the  $SP_3$  technique. Static results are then compared against transport benchmarks to assess the proximity of  $SP_3$ -ACMFD solutions to their full transport peers. Results prove that the approach can be considered as an acceptable interim approximation with outputs superior to the diffusion method, close to the transport results, and with the computational costs less than the full transport approach. The work would be further generalized to time dependent solutions in Part II.

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#### 1. Introduction

Accuracy of the neutron diffusion theory is not satisfactory for design and analysis of reactors with complex fuel assemblies e.g. MOX fueled LWRs [1]. Taking into account the rich phase space of transport equation, a time dependent analysis would be too expensive in the sense of computational cost. Among deterministic methods, discrete ordinates  $(S_N)$  or expansion through spherical harmonics polynomials  $(P_N)$  are the most favorable techniques to solve the transport equation. Although theoretical investigations predict a good convergence for large enough values of N, in practice it is not quite economic for core calculation [2]. To reduce the computational burden while saving the desired accuracy as much as possible, a number of efforts have focused on interim approaches which resulted in Simplified  $S_N(SS_N)$  as well as Simplified  $P_N(SP_N)$  for transport solutions [3-5]. These methods accompany the good precision of semi-analytic expansions alongside reducing the costs of computation through some acceptable simplifications for angular variables. The platform is therefore suitable for a time dependent analysis.

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Based on analytic investigations [6], the  $SP_N$  approximation allocates less variables compared to its  $P_N$  counterpart. As an instance, a 3D even parity  $P_3$  analysis requires six moments per node while the same analysis in  $SP_3$  becomes possible with only two moments per node suggesting an incredible saving.

In recent years many researchers have benefited  $SP_3$  equations for complicated cores. Bahabadi et al. [7] proposed a new analytic function expansion nodal (AFEN) method for solving  $SP_3$  static equations which uses analytic basis functions to expand 3D flux. So, it might be encountered as a novel expansion nodal method. More recently, the  $SP_N$  coupled with the finite element method was used for pinwise homogenization with less computational cost [8].

A rather quick scan over the works carried out on the  $SP_3$  approach decisively indicates a fast growth of interests in this plausible approximation. For instances, Chao [9] continues efforts on his proposed generalized  $SP_N$  ( $GSP_N$ ) equations already developed for a generic case of multi-group anisotropic neutron source with improved boundary conditions [10,11]. Though Chao derived some new and complicated interface and boundary conditions plus more differential equations to improve the accuracy of original  $SP_N$  equations, the work is not yet supported by detailed numerical experiments to prove the predicted error reduction in applied problems. In another attempt, Cherezov et al. [12] focused on the diffusion and  $SP_3$  equations for static 2D reactor core calculations

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using the reduced order element method. At the same time, Zhang and Chen [13] tried to solve the  $SP_3$  equations using the reduced basis finite element method. Meanwhile, the coupled diffusion/ $SP_3$  method was implemented in a so called GLIM framework using the Hybrid Nodal Green's Function Method (HNGFM) [14]. Finally, Yang et al. [15] experienced the accelerated pin-by-pin calculations using the exponential function expansion nodal  $SP_3$  solved by the GMRES technique applicable for PWRs.

At the time being, one can list many codes for static  $SP_3$  solution but few with transient ability, topped by PARCS [16] and DYN3D [6]. Both codes benefit the nodal expansion method (NEM) for the spatial discretization. The code SCOPE2 is equipped with a semi-analytic nodal method for PWR core analysis [17]. We can also point to the SIMULATE-4 [18] wherein a variant of semi-analytic 2D  $SP_3$ -nodal formulation is coupled with an axial homogenization model to carry out 3D core calculations.

Analytic coarse mesh finite difference (ACMFD) method was first introduced by Chao [19] to derive an explicit analytic expressions for the two-group effective diffusion coefficients in both 3D Cartesian and hexagonal geometries. Later, Aragonés et al. [20] expanded the idea to multigroup case by transforming the physical nodal equations to the so-called mathematical *modal* space followed by solving energy decoupled equations there. The solutions are transformed back into the nodal space resulting in a coupled system of equations with group fluxes as unknowns.

In this paper we propose the extention of using ACMFD methodology for the solution of static  $SP_3$  equations. A solver called SIMANOD is therefore developed based on the  $SP_3$ -ACMFD technique to carry out a number of static criticality calculations. To demonstrate the superiority of the adopted  $SP_3$  technique over the diffusion outcome, we have also developed a difusion solver (DIFANOD) based on the ACMFD method. This way one will be able to keep the spatial approximation fixed for both codes and focus on the angular improvements achieved by the  $SP_3$  kernel. Fianlly, to verify the code against transport results, a number of benchmarks are examined.

Here in Part I, static SP<sub>3</sub> equations in three dimensions are discussed, and satisfactory benchmarking which justify the superiority of new platform over the diffusion model is also proposed. This way, necessary initial conditions for transient problems can be generated through the solution of static eigenvalue equations solver. Then in Part II, the idea is generalized to time dependent equations where the numerical solution of transient problems as well as the accuracy achieved over diffusion peers are investigated there.

The rest of this article is organized as follows. In section 2, we propose the coupled  $SP_3$ -ACMFD approach adopted for reactor core static analysis. Section 3 is devoted to benchmark problems supporting the methodology described earlier. Therein, a number of reactor cores are studied at the static mode in detail. We finally sum up the work with a conclusion in section 4.

## 2. The SP<sub>3</sub>-ACMFD technique applied for static solutions

The primary idea behind development of the  $SP_N$  approximation was to simplify the application of heavily coupled  $P_N$  equations in multidimensional space. The appraoch was successful in damping the number of coupled equations down to N+1 in a 3D  $SP_N$  system from  $(N+1)^2$  for a full 3D  $P_N$  analysis ( $N \in \text{odd}$ ).

### 2.1. SP3 equations for angular approximation

Briefly, a  $SP_3$  system of equations is presented through the following coupled system [16],

$$\begin{split} &\nabla.\phi_{1g} + \Sigma_{rg}\phi_{0g} = S_{0g} = \sum_{g' \neq g}^{G} \Sigma_{sg'g}\phi_{0g'} + \frac{\chi_{g}}{k_{eff}} \sum_{g'=1}^{G} \nu \Sigma_{fg'}\phi_{0g'}, \\ &\frac{2}{3} \nabla \phi_{2g} + \frac{1}{3} \nabla \phi_{0g} + \Sigma_{trg}\phi_{1g} = 0, \\ &\frac{3}{5} \nabla.\phi_{3g} + \frac{2}{5} \nabla.\phi_{1g} + \Sigma_{tg}\phi_{2g} = 0, \\ &\frac{3}{7} \nabla \phi_{2g} + \Sigma_{tg}\phi_{3g} = 0, \end{split} \tag{1}$$

where all parameters have their conventional definitions used in the transport theory. Next, odd order moments are substituted by even moments obtained via rearrangement of equations. The result can be displayed in matrix form as [16].

$$\begin{bmatrix} -D_1 \nabla^2 + \Sigma_r & -2\Sigma_r \\ -\frac{2}{3} \Sigma_r & -D_3 \nabla^2 + \Sigma_{rt} \end{bmatrix}_g \cdot \begin{bmatrix} \Phi_0 \\ \phi_2 \end{bmatrix}_g = S_{0,g} \begin{bmatrix} 1 \\ -\frac{2}{3} \end{bmatrix}_g$$
 (2)

where

$$\Phi_0 \equiv \phi_0 + 2\phi_2, \quad \Sigma_{rt} \equiv \frac{5}{3} \Sigma_t + \frac{4}{3} \Sigma_r, \quad D_1 \equiv \frac{1}{3 \Sigma_{tr}}, \quad D_3 \equiv \frac{3}{7 \Sigma_t}.$$
(3)

Partial currents can also be introduced as a special combination of moments of surface fluxes and moments of net currents

$$j_{1}^{\pm} = \frac{1}{4} \Phi_{0s} \pm \frac{1}{2} J_{1} - \frac{3}{16} \phi_{2s}, \quad j_{3}^{\pm} = \frac{7}{16} \phi_{2s} \pm \frac{1}{2} J_{3} - \frac{1}{16} \Phi_{0s}. \tag{4}$$

where current moments are defined as.

$$I_1 = -D_1 \nabla \Phi_0, \ I_3 = -D_3 \nabla \phi_2.$$
 (5)

Theoretically, the solution of Eq. (2) determines the scalar flux  $(\phi_0)$  plus related quantities. As typical for all nodal methods, in the first step Eq. (2) is integrated over the volume of node m and dimensions  $h_u$  (u=x,y,z).

$$\begin{split} &\sum_{u=x,y,z} \left[ \left( J_{1,g,u}^{m_u+} - J_{1,g,u}^{m_u-} \right) \middle/ h_{u,m_u} \right] + \Sigma_{r,g} \overline{\Phi}_{0,g} - 2 \Sigma_{r,g} \overline{\phi}_2 \\ &- \sum_{g'=1}^G \Sigma_{g' \to g} \left( \overline{\Phi}_{0,g'} - 2 \overline{\phi}_{2,g'} \right) = \frac{\chi_g}{k_{eff}} \sum_{g'} \nu \Sigma_{fg'} \left( \overline{\Phi}_{0,g'} - 2 \overline{\phi}_{2,g'} \right), \\ &\sum_{u=x,y,z} \left[ \left( J_{3,g,u}^{m_u+} - J_{3,g,u}^{m_u-} \right) \middle/ h_{u,m_u} \right] + \Sigma_{rt,g} \overline{\phi}_{2,g} - \frac{2}{3} \Sigma_{r,g} \overline{\Phi}_{0,g} \\ &+ \frac{2}{3} \sum_{g' \neq g} \Sigma_{sg'g} \left( \overline{\Phi}_{0,g'} - 2 \overline{\phi}_{2,g'} \right) = -\frac{2}{3} \left( \frac{\chi_g}{k_{eff}} \sum_{g'} \nu \Sigma_{fg'} \left( \overline{\Phi}_{0,g'} - 2 \overline{\phi}_{2,g'} \right) \right). \end{split}$$

Establishing a relation between the average nodal surface currents and the average nodal moments would lead to an eigenvalue system of equations for which the ACMFD framework is further invoked to solve the problem.

## 2.2. ACMFD method for spatial descretization

As quite conventional in nodal methods, three transverse integrations over the  $SP_3$  equations have to be performed to generate three one dimensional equations with transverse leakage terms. The result would be three transverse integrated equations of the

form

$$\begin{bmatrix} -D_1 \frac{d^2}{du^2} + \Sigma_r & -2\Sigma_r \\ -\frac{2}{3}\Sigma_r & -D_3 \frac{d^2}{du^2} + \Sigma_{rt} \end{bmatrix}_g \begin{bmatrix} \phi_0 \\ \phi_2 \end{bmatrix}_g = S_{0,g} \begin{bmatrix} 1 \\ -\frac{2}{3} \end{bmatrix} - \begin{bmatrix} L_1 \\ L_3 \end{bmatrix}_g$$

where u = x, y, or z and,

$$\begin{split} L_1(u) &= \frac{1}{h_{\nu} h_{w}} \mathcal{J} - D_1 \left( \frac{\partial^2 \Phi_0}{\partial \nu^2} + \frac{\partial^2 \Phi_0}{\partial w^2} \right) d\nu dw ; \qquad \nu, w \\ &= x, y, z \ (\nu \neq w \neq u). \end{split} \tag{8}$$

A similar equation can also be derived for  $L_3(u)$  in Eq. (7) by substituting proper parameters. Eq. (7) can be expressed in a condensed form as

$$\left(\frac{d^2}{du^2} - A\right) |\varphi(u)\rangle = D^{-1}|L(u)\rangle. \tag{9}$$

where  $\mathbf{D}_u$  is a diagonal matrix containing  $D_{ig}$ ,  $|\varphi(u)\rangle = [\Phi_{0,g} \quad \phi_{2,g}]^{\mathrm{T}}$  and  $\mathbf{A}$  is defined by

$$A = \begin{bmatrix} A^{11} & A^{12} \\ A^{21} & A^{22} \end{bmatrix}_{2G \times 2G}, \quad A^{ij} = \begin{bmatrix} a^{ij}_{gg'} \end{bmatrix}_{G \times G},$$

$$a^{11}_{gg'} = \frac{1}{D_{1,g}} \begin{cases} \sum_{s,g' \to g} + \chi_g \nu \Sigma_{f,g'} / k_{eff} & g \neq g' \\ -\Sigma_{r,g} + \chi_g \nu \Sigma_{f,g} / k_{eff} & g = g' \end{cases},$$

$$a^{12}_{gg'} = \frac{-2}{D_{1,g}} \begin{cases} \sum_{s,g' \to g} + \chi_g \nu \Sigma_{f,g'} / k_{eff} & g \neq g' \\ -\Sigma_{r,g} + \chi_g \nu \Sigma_{f,g} / k_{eff} & g = g' \end{cases},$$

$$a^{21}_{gg'} = \frac{-2}{3D_{3,g}} \begin{cases} \sum_{s,g' \to g} + \chi_g \nu \Sigma_{f,g'} / k_{eff} & g \neq g' \\ 1.5 \left( \sum_{r,g} - \chi_g \nu \Sigma_{f,g} / k_{eff} \right) & g = g' \end{cases},$$

$$a^{22}_{gg'} = \frac{4}{3D_{3,g}} \begin{cases} \sum_{s,g' \to g} + \chi_g \nu \Sigma_{f,g'} / k_{eff} & g \neq g' \\ -3\sum_{rt,g} / 4 + \chi_g \nu \Sigma_{f,g} / k_{eff} & g \neq g' \end{cases}$$

$$-3\sum_{rt,g} / 4 + \chi_g \nu \Sigma_{f,g} / k_{eff} & g \neq g' \end{cases}$$

$$(10)$$

In this paper the group transverse leakage for the typical node m is approximated by a quadratic polynomial as

$$L_{m}^{i,g}(u) = c_{0,m}^{i,g} + u c_{1,m}^{i,g} + \left(u^{2} - \frac{h_{u,m}^{2}}{12}\right) c_{2,m}^{i,g};$$

$$i = 1, 3,$$

$$g = 1, 2, ..., G.$$
(11)

with  $h_{u,m}$  being the length of node m along the u direction. To find the expansion coefficients ( $c_{n,m}^{i,g}$ ; n=0,1,2), a curve fitting process is performed which establishes a relation among  $c_{n,m}^{i,g}$  and the amounts of average transverse leakages over node m and its two adjacents i.e. m-1 and m+1 in the same direction (currently supposed to be known). Therefore, one can write

$$\overline{L}_{u}^{m-1} = \frac{1}{h_{u,m-1}} \int_{-\frac{h_{u,m}}{2}-h_{u,m-1}}^{-\frac{h_{u,m}}{2}} L_{i,g}(u)du = c_{0,m}^{i} - (h_{u,m-1} + h_{u,m})c_{1,m}^{i}/2 + (h_{u,m-1} + h_{u,m})(2h_{u,m-1} + h_{u,m})c_{2,m}^{i}/6;$$

$$\overline{L}_{u}^{m} = \frac{1}{h_{u,m}} \int_{-\frac{h_{u,m}}{2}}^{+\frac{h_{u,m}}{2}} L_{i,g}(u)du = c_{0,m}^{i};$$

$$\overline{L}_{u}^{m+1} = \frac{1}{h_{u,m+1}} \int_{-\frac{h_{u,m}}{2}}^{+\frac{h_{u,m}}{2}+h_{u,m+1}} L_{i,g}(u)du = c_{0,m}^{i};$$

$$c_{0,m}^{i} + (h_{u,m+1} + h_{u,m})c_{1,m}^{i}/2 + (h_{u,m+1} + h_{u,m})(2h_{u,m+1} + h_{u,m})c_{2,m}^{i}/6;$$

$$u = x, y, z;$$

$$c_{0,m}^{i} + (h_{u,m+1} + h_{u,m})(2h_{u,m+1} + h_{u,m})c_{2,m}^{i}/6;$$

where  $\overline{L}_u$  for nodes m-1, m and m+1 are obtained through a straightforward 1D integration over the volume of each node leading to an algebraic summation of average transverse currents over the node boarders. As a result, the unknown  $|L(u)\rangle$  in Eq. (9) can be presented as a function of transverse currents which is assumed to be known thanks to the previous iterations during the iterative solution or an implicit approach, as an alternate solution strategy. Therefore,  $|L(u)\rangle$  in Eq. (9) is now supposed to be known and the dependency of lateral directions was removed.

Despite achievments in spatial simplicities, Eq. (9) still suffers from extensive group and moment couplings which must be resolved to enjoy the ACMFD with analytic solution. By benefiting the diagonalization procedure, the  $2 \times G$  system of coupled equations could be reduced into a set of  $2 \times G$  uncoupled equations. To this end, as performed by Lozano et al. [21] for the diffusion case, we can define some conversion matrices to shift over the so-called *modal* space wherein the equations could get decoupled. Using the matrix algebra we may have

$$A|\nu_{m}\rangle = \lambda_{m}|\nu_{m}\rangle \; ; \; m = 1, ..., 2G. \; R^{-1} = [ \times |\nu_{1}\rangle \; |\nu_{2}\rangle \; ... \; |\nu_{2G}\rangle]; \; RAR^{-1} = \lambda,$$
 (13)

where  $|v_m\rangle$  represents the eigenvector of **A**, and  $\lambda$  is a diagonal matrix for eigenvalues of **A**. Nodal vectors could be transferred into the modal spaces through the conversion matrices as

$$|\psi(u)\rangle = R|\varphi(u)\rangle, \quad |\varrho(u)\rangle = RD^{-1}|L(u)\rangle$$
 (14)

and inverse transforms could be used for modal to nodal shift. Using transforms given by (14), Eq. (9) can be represented in the modal space as

$$\frac{d^{2}\psi_{m}(u)}{du^{2}} - \mu_{m}\psi_{m}(u) = \ell_{m}(u); \quad m = 1, ..., M,$$

$$u = x, y, z \tag{15}$$

with the following solution:

$$\psi_m(u) = P_m(u) + A_{m,u}e^{+\sqrt{\mu_m}u} + B_{m,u}e^{-\sqrt{\mu_m}u}$$
(16)

where  $P_m(u)$  is the particular solution corresponding to the specific force term  $\ell_m(u)$  at the rhs of Eq. (15), and  $A_{m,u}$  and  $B_{m,u}$  are two constants obtained with the help of boundary conditions. The general modal solution derived in Eq. (16) could be specialized for a particular 1D node expanded from u=-h/2 to u=+h/2. In this regard, some mathematical manipulation ends to a relation among the mean modal flux over the node, modal currents and flux moments on the node borders as

$$\psi_{m}\left(\pm\frac{h_{u}}{2}\right) = P_{m}\left(\pm\frac{h_{u}}{2}\right) + \xi_{m,u}^{f}\left[\overline{\psi}_{m} - \overline{P}_{m}\right]$$

$$\pm\frac{h_{u}\xi_{m,u}^{j}}{2}\frac{d}{du}\left[\psi_{m}\left(\pm\frac{h_{u}}{2}\right) - P_{m}\left(\pm\frac{h_{u}}{2}\right)\right]$$

$$m = 1 \quad M \quad u = x, y, z$$

$$(17)$$

where

$$\xi_{m,u}^{f} \equiv \frac{\sqrt{\mu_{m}}h_{u}}{\sinh(\sqrt{\mu_{m}}h_{u})}, \quad \xi_{m,u}^{f} \equiv \frac{\tanh(\sqrt{\mu_{m}}h_{u}/2)}{\sqrt{\mu_{m}}h_{u}/2}$$
 (18)

are defined for simplicity. Now, noting Eq. (9), the decoupled equations stated by Eq. (17) are transferred back into the nodal space to have a system of coupled equations there. The coupled nodal system can now be represented as

$$\left| \varphi_{u} \left( \pm \frac{h_{u}}{2} \right) \right\rangle = \Gamma_{u}^{f} |\overline{\varphi}\rangle \mp \frac{h_{u}}{2} \Gamma_{u}^{j} D^{-1} \left| J_{u} \left( \pm \frac{h_{u}}{2} \right) \right\rangle - R^{-1} \left| \Theta \left( \pm \frac{h_{u}}{2} \right) \right\rangle$$

$$u = x, y, z \tag{19}$$

where  $\left|\Theta\left(\pm\frac{h_u}{2}\right)\right>$  is the vector of  $\theta_{m,u}\left(\pm\frac{h_u}{2}\right)$  elements for m= 1,2, ...,2G as

$$\theta_{m,u}\left(\pm\frac{h_u}{2}\right) = \xi_{m,u}^f \overline{P}_{m,u} - P_{m,u}\left(\pm\frac{h_u}{2}\right) \pm\frac{h_u}{2} \xi_{m,u}^j \frac{d}{du} P_{m,u}\left(\pm\frac{h_u}{2}\right)$$
(20)

and

$$\Gamma_u^f = R^{-1} \Xi_u^f R, \quad \Gamma_u^j = R^{-1} \Xi_u^j R \tag{21}$$

are two coefficient matrices wherein  $\Xi$  denote diagonal matrices for  $\xi$ s. Imposing continuity conditions for the moments and currents at the interfaces of samples nodes m-1 and m establishes a relation between  $|\overline{J}_{u,L}^{m_u}\rangle$  and  $|\overline{\varphi}_u\rangle$  for the two nodes as

$$\begin{split} \left| \overline{J}_{u,L}^{m_u} \right\rangle &= \frac{1}{2} \left( h_u^{m_u - 1} \Gamma_u^{j,m_u - 1} \left[ D^{m_u - 1} \right]^{-1} + h_u^{m_u} \Gamma_u^{j,m_u} \left[ D^{m_u} \right]^{-1} \right)^{-1} \\ \left( \Gamma_u^{f,m_u - 1} \left| \overline{\varphi}^{m_u - 1} \right\rangle - \left[ R^{m_u - 1} \right]^{-1} \left| \Theta_{u,R}^{m_u - 1} \right\rangle - \Gamma_u^{f,m_u} \left| \overline{\varphi}^{m_u} \right\rangle \\ + \left[ R^{m_u} \right]^{-1} \left| \Theta_{u,L}^{m_u} \right\rangle \right), \ u = x, y, z; \ L : left, \ R : right \end{split}$$

$$(22)$$

Finally, to characterize the equations, two external boundary conditions must be further incorporated. For the node M subjected to a reflective surface the current  $|\bar{J}_{u}^{M}\rangle$  damps to zero at the boundary, and for the node M' with free surface, the net partial incoming current in Eq. (4) vanishes by force. This is performed through setting

$$\left| J_{u}^{M'} \left( \pm \frac{h_{u,FS}}{2} \right) \right\rangle = \pm B \left| \varphi_{u}^{M'} \left( \pm \frac{h_{u,FS}}{2} \right) \right\rangle, \quad B = \frac{1}{8} \begin{bmatrix} 4 & -3 \\ -1 & 7 \end{bmatrix}, \quad FS$$

: Free Surface.

(23)

for all energy groups of node M' where "+" goes for the right free surface while "-" comes with the left free surface.

Finally, substituting the terms of currents stated by Eq. (22) for different  $J^{m_u\pm}_{i,g,u}$  terms into the nodal balance Eq. (6) plus incorporating external boundary conditions, renders a multigroup eigenvalue system of the form  $\mathbf{M}|\overline{\varphi}\rangle=\frac{1}{k_{eff}}\mathbf{F}|\overline{\varphi}\rangle$  for the static state. The commonly invoked power iteration can in general solve the problem straightforwardly with transverse leakage updating included. Fig. 1 presents a calculation flow for the solution of equations described earlier.

#### 3. Numerical results

Based on the theory, a code named SIMANOD is developed in F90 to perform 3D multigroup static solutions benefiting the *SP*<sub>3</sub>-ACMFD scheme. The engine was also boosted by the Math Kernel Library (MKL). The powerful GMRES solver is invoked to solve the final linear system of the equations.

Four benchmark problems are chosen for the evaluation of static results. According to the theory, we expect superior outcome compared to the diffusion predition especially in problems with high heterogenuity. To prove it, we have further developed a code called DIFANOD based on the ACMFD but for the difussion analysis. On the other hand, the theory states that the  $SP_3$  should result in inferoir results than the transport solutions of course with lower costs. For complex problems however, the privilege balances toward the interim approaches ( $SP_3$  included) as the solution costs must not exceed a logical threshold. Compartive outcomes could be traced in the supplied static benchmarks. The convergence criteria for  $k_{\rm eff}$  and the local flux are  $10^{-7}$  and  $10^{-4}$  respectively for all tests

## 3.1. Test Case I: a small homogeneous cube

As the first, we consider a simple homogeneous cube of 10 cm in each side. The cube dimension is small enough to observe cosiderable anisotropy of the neutron flux as a result of high boundary to volume ratio. Geometry of the problem is demonstrated in Fig. 2 wherein a few boundary nodes are also specified. Also, one group cross sections are  $(\Sigma_t, \nu \Sigma_f, \Sigma_s) = (1, 0.88, 0.25)$  cm<sup>-1</sup>. Cubic nodes of unit volume fill the cube for both SIMANOD and DIFANOD.

As the reference, MCNPX [22] results for the  $k_{eff}$  is reported with 1 pcm standard deviation, and the maximum relative standard deviation for the fluxes is 0.0006.

In Eq. (24) we have introduced the Relative Power Density (RPD) for node i which is applicable for the comparison of power distibution. In fact, RPD<sup>i</sup> is the ratio of power density in node i to mean power density of the core.

$$\mathsf{RPD}^i = \frac{\sum\limits_{g=1}^G \Sigma_{f,g}^i \overline{\phi}_{0,g}^i}{\left(\sum\limits_{i=1}^N V^i \sum\limits_{g=1}^G \Sigma_{f,g}^i \overline{\phi}_{0,g}^i\right) \bigg/ \left(\sum\limits_{i=1}^N \lambda^i V^i\right)},$$

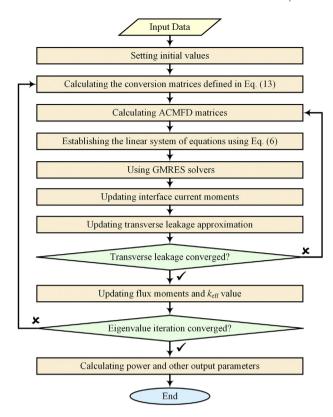


Fig. 1. A calculation flow diagram for the static SP<sub>3</sub>-ACMFD approach.

$$\lambda^{i} = \begin{cases} 1 \; ; \; \; \Sigma_{f,g}^{i} \neq 0 \\ 0 \; ; \; \; \Sigma_{f,g}^{i} = 0 \end{cases}$$
 (24)

Table 1 demonstarates that switching from DIFANOD to SIMANOD i.e. diffusion to  $SP_3$  improves the error in the effective multiplication factor from -543 to +43 pcm. Also in Table 1, the improvement is quite sensible (nearly 8 times) in the average of absolute errors in RPD over the whole core. By moving from the cube center to the wall, the flux anisotropy is intensified and consequently the need for finer angular resolution is much felt. To understand the maximum improvement obtained by SIMANOD against DIFANOD, the RPD is monitored over the boundary nodes depicted in Fig. 2. As expected, corner nodes pose the maximum error due to the high flux gradient there. Via Fig. 3, one can confirm that the approach adopted in SIMANOD nearly halves the absolute error of this node compared to that of DIFANOD.

#### 3.2. Test Case II: a 2D LWR with UOX-MOX fuels (C5G2)

To illustrate the capabilities of the developed codes in analyzing the MOX fuel cores, this two-dimensional two-group problem is studied in detail. Capilla et al. [23] introduced the problem as a simplified version of the famous C5G7 core originally proposed in Ref. [24]. The core configuration is depicted in Fig. 4 and the two-group constants of the problem can be found in Ref. [23]. Each assembly must be divided into at least  $17 \times 17$  rectangles with the minimum of 1156 nodes in the active core. Nodes of variable size however could be employed for the surrounding homogeneous reflector.

To assess the proposed approach over this highly heterogeneous benchmark, the minimum number of required nodes are used for discretizing the core. Also, to improve the resolution for the

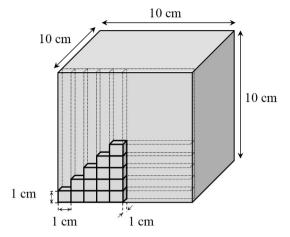


Fig. 2. Geometry of the simple homogeneous cube (Test Case I).

**Table 1**Eigenvalue search results for the simple homogeneous cube (Test Case I).

CODE	$k_{ m eff}$	$\varepsilon_k^a$ [pcm]	ε <sub>ave</sub> <sup>b</sup> [%]	ε <sub>max</sub> <sup>b</sup> [%]
MCNPX (ref.)	1.06938	0	0	0
SIMANOD	1.06981	+43	0.22	1.99
DIFANOD	1.06395	-543	1.76	4.62

<sup>&</sup>lt;sup>a</sup>  $\varepsilon_k$  is the error in effective multiplication factor in comparison with the reference value (pcm).

thermal group, nodes of the same size are tiled over the water reflector. As a result the problem comprises 2601 nodes in sum. Reference solution of the benchmark is generated by MCNPX to facilitate a detail comparison for pin powers as well as the eigenvalue. Again, MCNPX results for the  $k_{\rm eff}$  is reported with 1 pcm standard deviation, and the maximum relative standard deviations for pinwise fluxes are 0.0006 and 0.0008 for the fast and thermal energy groups, respectively. Eigenvalue search results plus calculated errors are reported in Table 2. Moreover, to see the improvement obtained over the diffusion approximation results by the DIFANOD are also reported there. Absolute values of pin power distribution error over the assemblies are further compared in Fig. 5.

Table 2 displays a relatively low-cost switch from the commonly invoked diffusion method to the  $SP_3$  approach which deserves a reduction in the error of  $k_{\rm eff}$  from 90 to 31 pcm (nearly 3 times lower). The gain in pin powers are also brilliant. The maximum error of pin powers is reduced by a factor of 5 while in average, the

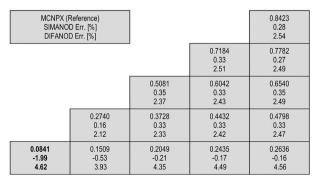


Fig. 3. RPD results for the boundary nodes shown in Fig. 2 (Test Case I).

 $<sup>^{\</sup>rm b}$   $\epsilon_{ave}$  and  $\epsilon_{\rm max}$  are average and maximum of the absolute percentage of relative errors in nodal power densities.

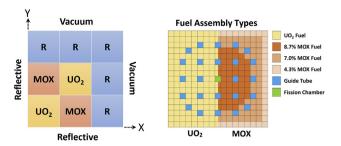
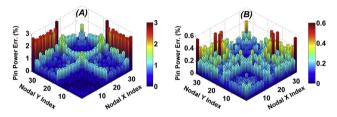


Fig. 4. Geometry of the C5G2 benchmark (Test Case II).

 Table 2

 Eigenvalue search results for the C5G2 benchmark (Test Case II).

Code	$k_{ m eff}$	$\varepsilon_k[pcm]$	$\varepsilon_{ave}[\%]$	$\varepsilon_{\text{max}}[\%]$
MCNPX	0.97050	0	0	0
SIMANOD	0.97081	+31	0.13	0.56
DIFANOD	0.97140	+90	0.49	2.95



**Fig. 5.** Pin power errors in the C5G2 benchmark using (A) DIFANOD, and (B) SIMANOD (Test Case II).

errors are at least dropped by a factor of 3. The map of pin power errors are further illustrated in Fig. 5 where as demonstrated, maximum errors are appeared near the borders.

## 3.3. Test Case III: the modified IAEA-3D problem

As a 3D test, the simplified IAEA's PWR is investigated to assess the performance of SIMANOD under normal operation. The problem was first introduced by Hébert [3] who modified the original core proposed in Ref. [25]. The core specification and the two group constants are the same as those reported in Ref. [3].

Cho et al. [26] modeled the problem using the code DANTSYS with  $S_{16}$  approximation which is taken as reference. For our purpose, we axially subdivided each assembly into 19 nodes  $(1 \times 1 \times 19)$  to obtain the results. The multiplication factor and the relative powers of the assemblies are compared against the reference results in Table 3 and Fig. 6 respectively. Again, a substantial improvement comes along the expectations when shifting over the  $SP_3$  equations. As seen in Table 3, the eigenvalue error comes down to 2 pcm from 48 pcm, and both  $\varepsilon_{ave}$  and  $\varepsilon_{max}$  experience an improvement by a factor of nearly four. Note that the power errors in Table 3 are defined over assemblies. Meanwhile, Bahabadi et al. [7] have supposed 76  $(2 \times 2 \times 19)$  nodes per assembly for a  $SP_3$ -AFEN analysis using their MGANSP3 code. Results are very close to

**Table 3**Eigenvalue search results for the modified IAEA-3D problem (Test Case III).

Method	$k_{ m eff}$	$\varepsilon_k$	$\varepsilon_{ave}$	$\varepsilon_{max}$
DANTSYS (S16) SIMANOD	1.02956 1.02958	_ 2	- 0.19	- 0.68
DIFANOD	1.02908	48	0.72	2.91

	S <sub>16</sub> (Reference) SIMANOD Err. [%] DIFANOD Err. [%]				0.608 -0.12 -1.46			
				•	0.469 -0.15 1.30	0.71 0.68 -0.70	0.621 0.55 -0.87	
			1.17 0.03 0.45	0.97 0.02 0.13	0.93 -0.13 -0.58	0.88 -0.02 -1.06		
			1.352 -0.21 0.73	1.298 0.13 0.76	1.176 -0.09 0.21	1.091 -0.09 -0.24	1.013 -0.09 -1.17	0.725 -0.21 -1.37
		1.377 -0.11 0.77	1.415 -0.21 0.66	1.278 -0.14 0.52	1.064 0.15 0.50	1.056 -0.17 -0.25	0.983 -0.16 -0.67	0.768 0.44 -0.87
	0.708 0.17 2.91	1.265 -0.36 0.54	1.406 -0.46 0.41	1.178 0.06 0.77	0.597 0.00 2.18	0.953 0.06 0.00	0.965 -0.02 -0.48	0.788 0.23 -1.04

**Fig. 6.** Relative power of the assemblies for the modified IAEA-3D problem (Test Case III).

those we obtained but in comparison, the absolute error in  $k_{\rm eff}$  and the maximum error in relative powers decrease by 14 pcm and 0.77%, respectively.

#### 3.4. Test Case IV: Takeda's small fast breeder reactor

As the last problem, a small fast breeder reactor (FBR-SQ) is chosen among famous benchmark problems of Takeda and Ikeda [27]. Two different cases of the problem are considered: (i) uncontrolled, where control channel is filled by the sodium coolant, and (ii) partially controlled, where the control rod is half inserted. The reference results are adopted from the work of Ryu and Joo [28] who employed the Monte Carlo code McCARD. The size of nodes in both SIMANOD and DIFANOD, are assumed  $5 \times 5 \times 5$  cm<sup>3</sup>. Table 4 lists a comparison among evaluated parameters. Also, relative power distributions of cases i and ii are compared in Fig. 7 for core region. Four group nodal fluxes of the core's fifth layer (from the top) are depicted in Fig. 8. The flux is normalized such that the neutron generation rate equals unity over the full core.

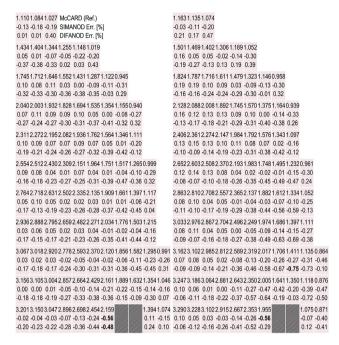
Overall, inspection of Table 4 reveals that the error of proposed method lies in the acceptable range for both conditions, and for the partially controlled case, the eigenvalue errors are generally more for uncontrolled state, as expected. The maximum error however remains unchanged for SIMANOD. It is also observed that the eigenvalue errors are reduced by an order of magnitude when switching from SIMANOD to DIFANOD beside a considerable reduction in the mean values of power errors. The only drawback is the maximum error of power in uncontrolled case which goes up by

Over the detailed comparisons given in Fig. 7, one should note that the reference values of McCARD have been reported only with fixed 3 digits after point. Therefore, numerical comparisons made in Fig. 7 may not be entirely true especially inside the blanket where the number of reference meaningful digits may be reduced down to one. That's why we have only reported the maximum relative error in power for the core region.

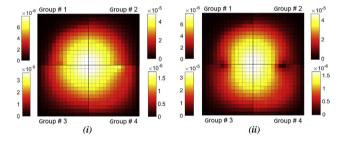
**Table 4**Eigenvalue search results for Takeda's small fast breeder reactor benchmark (Test Case IV).

Code	Case	$k_{\rm eff}$	$\epsilon_k[pcm]$	ε <sub>ave</sub> <sup>a</sup> [%]	ε <sub>max</sub> a [%]
McCARD (Ref.) Ryu and Joo [28]	Uncontrolled Partially controlled	0.97370 0.95967	0	0	0 0
SIMANOD	Uncontrolled Partially controlled	0.97342 0.95926		0.09 0.14	0.56 0.56
DIFANOD	Uncontrolled Partially controlled	0.96887 0.95409	-480 -558	0.27 0.28	0.48 0.75

<sup>&</sup>lt;sup>a</sup> Errors are considered only in the core region.



**Fig. 7.** Power distribution for Takeda's small fast breeder reactor benchmark, cases i and ii (Test Case IV).



**Fig. 8.** Four-group scalar flux distribution for Takeda's small fast breeder reactor benchmark, cases i and ii (Test Case IV).

#### 4. Conclusion

A coupled  $SP_3$ -ACMFD technique was proposed for reactor core analysis. The main privilege of the idea is the admissible accuracy compared to the diffusion approximation while less burden in comparison with  $P_3$  resolution. To validate the idea a code called SIMANOD was developed for the static analysis of reactors with rectangular geometry. The approach proved robust for handling regular multigroup calculations. Simulations confirm that while the method is generally superior to the diffusion theory (DIFANOD), the overall accuracy is comparable with full transport approach. Static analyses suggest that the methodology can hopefully serve as an efficient platform for a further extension of the work to time dependent problems; an idea to be probed in Part II.

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#### Appendix A. Supplementary data

Supplementary data to this article can be found online at https://doi.org/10.1016/j.net.2019.07.022.

#### References

- S.A. Thompson, Advanced Reactor Physics Methods for Heterogeneous Reactor Cores, The pennsylvania state university college of engineering, 2014.
- [2] A. Henry, Nuclear-Reactor Analysis, second ed., MIT Press, 1980.
- [3] A. Hébert, Mixed-dual implementations of the simplified Pn method, Ann. Nucl. Energy 37 (2010) 498–511.
- [4] P. Kotiluoto, Adaptive Tree Multigrids and Simplified Spherical Harmonics Approximation in Deterministic Neutral and Charged Particle Transport, VIT Publications, 2007.
- [5] G. Longoni, A. Haghighat, The even-parity simplified S<sub>N</sub> equations applied to a MOX fuel assembly benchmark problem on distributed memory environments. PHYSOR 2004-the physics of fuel cycles and advanced nuclear systems, Global Developments (2004) 25–29.
- [6] C. Beckert, U. Grundmann, Development and verification of a nodal approach for solving the multigroup SP3 equations, Ann. Nucl. Energy 35 (2008) 75–86.
- [7] M.H.J. Bahabadi, A. Pazirandeh, M. Athari, New analytic function expansion nodal (AFEN) method for solving multigroup neutron simplified P<sub>3</sub> (SP<sub>3</sub>) equations, Ann. Nucl. Energy 77 (2015) 148–160.
- [8] A. Vidal-Ferràndiz, S. González-Pintor, D. Ginestar, C. Demazière, G. Verdú, Pin-wise homogenization for SPN neutron transport approximation using the finite element method, J. Comput. Appl. Math. 330 (2017) 806–821.
- [9] Y.-A. Chao, A new and rigorous SP<sub>N</sub> theory—Part III: a succinct summary of the GSP<sub>N</sub> theory, the P<sub>3</sub> equivalent and implementation issues, Ann. Nucl. Energy 119 (2018) 310–321.
- [10] Y.-A. Chao, A new and rigorous  $SP_{\rm N}$  theory for piecewise homogeneous regions, Ann. Nucl. Energy 96 (2016) 112–125.
- [11] Y.-A. Chao, A new and rigorous SP<sub>N</sub> theory—Part II: generalization to GSP<sub>N</sub>, Ann. Nucl. Energy 110 (2017) 1176—1196.
- [12] A. Cherezov, R. Sanchez, H.G. Joo, A reduced-basis element method for pin-bypin reactor core calculations in diffusion and SP<sub>3</sub> approximations, Ann. Nucl. Energy 116 (2018) 195–209.
- [13] C. Zhang, G. Chen, Fast solution of neutron transport SP<sub>3</sub> equation by reduced basis finite element method, Ann. Nucl. Energy 120 (2018) 707–714.
- [14] T.-Y. Lin, Y.-W.H. Liu, A next generation method for light water reactor core analysis by using global/local iteration method with SP<sub>3</sub>, Ann. Nucl. Energy 118 (2018) 49–60.
- [15] W. Yang, H. Wu, Y. Li, L. Cao, S. Wang, Acceleration of the exponential function expansion nodal SP3 method by multi-group GMRES algorithm for PWR pinby-pin calculation, Ann. Nucl. Energy 120 (2018) 869–879.
- [16] T. Downar, D. Lee, Y. Xu, T. Kozlowski, User and Theory Manual for the PARCS Neutronics Core Simulator (U.S. NRC Core Neutronics Simulator), School of Nuclear Engineering Purdue University, 2004.
- [17] M. Tatsumi, A. Yamamoto, Advanced PWR core calculation based on multigroup nodal-transport method in three-dimensional pin-by-pin geometry, J. Nucl. Sci. Technol. 40 (2003) 376–387.
- [18] T. Bahadir, S. Lindahl, S.P. Palmtag, SIMULATE-4 multi-group nodal code with microscopic depletion model, in: American Nuclear Society Topical Meeting in Mathematics and Computations, Avignon, France, 2005.
- [19] Y.A. Chao, A Theoretical Analysis of the Coarse Mesh Finite Fifference Representation in Advanced Nodal Methods. Mathematics and Computation, Reactor Physics and Environmental Analysis in Nuclear Applications, Senda Ed., 1999, pp. 117–126. Madrid.
- [20] J.M. Aragonés, C. Ahnert, N. García-Herranz, The analytic coarse mesh finite difference method for multigroup and multidimensional diffusion calculations, Nucl. Sci. Eng. 157 (2007) 1–15.
- [21] J.-A. Lozano, N. García-Herranz, C. Ahnert, J.-M. Aragonés, The analytic nodal diffusion solver ANDES in multigroups for 3D rectangular geometry: development and performance analysis, Ann. Nucl. Energy 35 (2008) 2365–2374.
- [22] M.U.s. Manual, Version 2.7. 0, Los Alamos National Laboratory, Los Alamos (NM), 2011.
- [23] M. Capilla, D. Ginestar, G. Verdú, Applications of the multidimensional P<sub>L</sub> equations to complex fuel assembly problems, Ann. Nucl. Energy 36 (2009) 1624–1634.
- [24] OECD/NEA, Benchmark on Deterministic Transport Calculations without Spatial Homogenisation: a 2-D/3-D MOX Fuel Assembly Benchmark. NEA/ NSA/DOC(2003)16, 2003.
- [25] Argonne Code Center, ANL Benchmark Book-Report ANL-7416, Argonne National Laboratory, Argonne, IL, 1977.
- [26] B. Cho, J.H. Won, N.Z. Cho, Analytic function expansion nodal(AFEN) method extended to multigroup simplified P<sub>3</sub> (SP3) equations via partial current moment transformation, Trans. Am. Nucl. Soc. 103 (2011) 714–717.
- [27] T. Takeda, H. Ikeda, 3-D neutron transport benchmarks, J. Nucl. Sci. Technol. 28 (1991) 656–669.
- [28] E.H. Ryu, H.G. Joo, Finite element method solution of the simplified  $P_3$  equations for general geometry applications, Ann. Nucl. Energy 56 (2013) 194-207.