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Original Article

Modeling and experimental production yield of ⁶⁴Cu with ^{nat}Cu and ^{nat}Cu-NPs in Tehran Research Reactor



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ABSTRACT

 64 Cu is a favorable radionuclide in nuclear medicine applications because of its unique characteristics such as three types of decay (electron capture, β^- and β^+) and 12.7 h half-life. Production of 64 Cu by irradiation nat Cu and nat CuNPs in Tehran Research Reactor was investigated. The characteristics of copper nanoparticles were investigated with SEM, TEM and XRD analysis. The cross section of 63 Cu(n,γ) 64 Cu reaction was done with TALYS-1.8 code. The activity value of 64 Cu was calculated with theoretical approach and MCNPX-2.6 code. The results were compared with related experimental results which showed good adaptations between them.

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1. Introduction

The 64 Cu is a unique radionuclide ($T_{1/2}=12.7$ h) with decay mode of E.C. (43.53%), β^- (38.48%) and β^+ (17.52%); End-point $E_{\beta}^+=653$ keV, End-point $E_{\beta}^-=579$ keV, $E_{\gamma}=1345.84$ keV, $I_{\gamma}=0.47\%$ (Fig. 1). According to these properties, 64 Cu is described as a useful radionuclide for both high resolution Positron Emission Tomography (PET) imaging and targeted endoradiotherapy. When this radionuclide is deposited in the cell, its electron capture decay can be more efficient in cell killing [1].

⁶⁴Cu can be produced with nat Cu(n,γ)⁶⁴Cu and 63 Cu(n,γ)⁶⁴Cu reactions in nuclear reactor [3–5]. The advantages of this reaction are abundant thermal neutrons and large cross sections. In 2010, the cross section of this reaction was reported by Cohen and et al. about 4.5 ± 0.1 b via thermal neutron induce reaction respectively [5]. These values are reasonably high for medical applications in comparison with some other radionuclides which can be produced with thermal neutron such as 123 Sn ($\sigma_{th} = 0.183$ b), 103 Ru ($\sigma_{th} = 1.23$ b), 113 Sn ($\sigma_{th} = 1.14$ b), 141 Ce ($\sigma_{th} = 0.575$ b) and 197 Pt ($\sigma_{th} = 0.74$ b) [6]. The Disadvantages are no simple chemical separation and low specific activity [7], while use of fast neutrons are

compulsory for high specific activity of 64 Cu via 64 Zn(n,p) 64 Cu reaction [3].

The Application of nanoparticles on ⁶⁴Cu production has its objective. The most important characteristic of radioactive nanoparticles is its ability to have several radioactive atoms in one single nanoparticle [8]. In our previous work, we researched on ⁶⁴Cu production with ^{nat}ZnO and ^{nat}ZnONPs targets via ^{nat}Zn(n,p)⁶⁴Cu reaction. We concluded that the activity of ⁶⁴Cu with nano scale target was increased [9]. Other researches in this subject were ¹⁹⁸Au, ¹⁴¹Ce, ¹⁴²Pr radionuclides productions [8,10–12].

In this research 64 Cu was produced with 1 g of nat CuNPs and nat Cu powders with natural abundance via the 63 Cu(n, γ) 64 Cu reaction in Tehran Research Reactor (TRR). The activity of 64 Cu from bulk and nano scale targets at the End of Bombardment (EOB) was compared with each other. The Transmission Electronic Microscopy (TEM), Scanning Electronic Microscopy (SEM) and X-Ray Diffraction (XRD) analysis for determining nat CuNPs characteristics before and after irradiation were done. The theoretical activity of 64 Cu was calculated by using the MCNPX and TALYS codes and was compared with corresponding experimental value.

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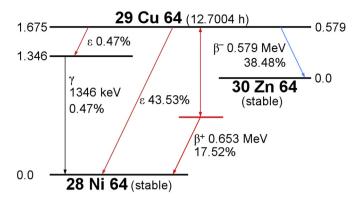


Fig. 1. The decay scheme of the ⁶⁴Cu radionuclide [2].

2. Material and methods

2.1. Material

The natural copper has two stable isotopes which are ⁶³Cu with abundance of 69.17% and ⁶⁵Cu with abundance of 30.83%. All chemical powders were purchased from US Research Nanomaterials, Inc. (Houston, USA). Physical properties and chemical composition of powders are presented in Table 1.

2.2. Theoretical activity calculation

The activity of the product can be calculated with the following equation [13]:

$$A = N. \sigma. \Phi. \left(1 - e^{-\lambda t_{irr}}\right). D_C \tag{1}$$

Where A is the activity in Bq after an irradiation period of t_{irr} , $N = \frac{m \cdot H_i \cdot N_A}{M}$ is the number of target's atom with the weight of target (m), abundance of 63 Cu (H_i) , Avogadro's Number (N_A) and molar mass of target (M), σ is the thermal neutron capture cross section, Φ is the thermal neutron flux, λ is the decay constant of 64 Cu and $D_C = e^{-\lambda T}$ is a constant for determining the activity after cool down period (T). The activity of target under irradiation process is increased with exponential factor in Eq. (1) and reaches maximum saturation value, but the activity of target after irradiation time is decreased with exponential factor in the D_C constant.

2.3. Cross section calculation

In this investigation, cross sections of 63 Cu(n, γ) 64 Cu reaction was calculated with TALYS-1.8 nuclear code. The results were compared with ENDF/B-VII.1 and experimental data which are available in EXFOR library. The ENDF/B-VII.1 library is theoretical

nuclear data for using in nuclear science and technology applications [14]. TALYS-1.8 is the latest update version which is efficient to calculate nuclear reaction cross section [15–19]. Neutrons, photons, protons, deuterons, tritons, ^3He , and $\alpha\text{-particles}$ can be used as a projectile on target nuclei in this code with selective energy range of 1 keV–200 MeV. Several options are included such as $\gamma\text{-ray}$ strength functions, nuclear level densities, nuclear model parameters and etc. while resonance parameter were used in the abovementioned reaction as recent parameters in the latest version of TALYS code [20–24].

2.4. Description of irradiation process

The 64 Cu was produced with 1 g of $^{\rm nat}$ Cu and $^{\rm nat}$ CuNPs powders respectively. The powders were taken in quartz ampules and then flame sealed separately and was placed inside the aluminum cans for irradiating in the core of TRR. The power of reactor in this study was 4 MW with approximately thermal neutron flux of 4.5×10^{13} n.cm $^{-2}$.s $^{-1}$ in the samples placement. The irradiation time for targets was 32 min and after 2 days cooling time two targets were dissolved in 20 ml 10 M HNO $_3$ and were diluted. The chemical processing of the irradiated targets were carried out as making homogenous solution and reasonable activity for gamma spectrometry. The production yield was estimated with a high purity ptype detector (HPGe) coupled with a multichannel analyzer (MCA) (Silena International, Rome, Italy) with a relative efficiency of 37% which calibrated with a point-like source of 152 Eu manufactured by Amersham Company.

2.5. Copper nanoparticles characteristics

The characteristics of copper nanoparticles were done with XRD from US Research Nanomaterials, Inc. (Houston, USA) when the nanoscale sample was not placed in the vicinity of the air. The chemical composition of ^{nat}CuNPs with XRD from (STOE STADI, MP, Germany) were done after 6 months when it was placed in the vicinity of the air. The XRD analysis is used for the crystallite size and structure of samples. The crystallite size was calculated with the Scherrer's equation (2) at a scanning rate from 5 to 90 [25].

Crystallite size (average) =
$$\frac{0.89 \,\lambda}{B \cos \theta}$$
 (2)

Where 0.89 is Scherrer's constant for sphere particles, λ is the wavelength of X-Rays, θ is the Bragg diffraction angle, and β is the full width at half-maximum (FWHM) of the diffraction peak corresponding to plane. Morphology of the samples were investigated by using TEM and SEM analysis from Iranian Nanomaterials Pioneers Company, NANOSANY (Mashhad, Iran). After irradiation process, SEM images of copper nanoparticles were performed with Digital SEM, EM 3200 KYKY.

Table 1 Physical properties and chemical composition of ^{nat}CuNPs.

Physical properties											
Purity (%)	Average part	ticle size (nm)	Specific surface	area (m² g ⁻¹)	True density (g c	m ⁻³) Mor	phology	color			
99.5	20	20		30-45		sphe	erical	Saddle brown			
Chemical com	position (%)										
Cu	As	Sb	Pb	Sn	Fe	Ni	Bi	0			
≥99.5	≤0.002	≤0.002	≤0.001	≤0.009	≤0.006	≤0.102	≤0.002	1.36			

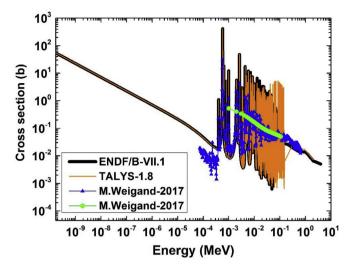


Fig. 2. Cross section of 63 Cu(n, γ) 64 Cu reaction calculated by TALYS-1.8 codes along with experimental and theoretical values.

2.6. Simulation method based on MCNPX code

The MCNPX code was selected to simulate the core of TRR and defining the production yield of radionuclides. The TRR is a maximum power of 5 MW (in this work 4 MW), pool type with 9 \times 6 grid plate for insertion 28 sets of fuel plates, 4 control rods, a shim safety rod, 14 packages of graphite box and 7 irradiation boxes. The fuel plates contain low enriched uranium fuel plates in the form of U₃O₈Al alloy. The reactor pool contains two sections, one of them is narrow stall end and other section is open end. The narrow stall end contains experimental facilities like beam tubes, rabbit system and thermal column. The open end section was designed for bulk irradiation [26–28].

In this methods, to calculate the 64 Cu production yield, The natural copper powder inside quartz ampoule and aluminum can located at the irradiation box of reactor core were described in our previous work [9,10]. The thermal neutron energy distribution $\varphi(E)$ at the $^{\text{nat}}$ Cu target with F4/E4 cards-MCNPX was calculated and by the results of TALYS-1.8 code the below integration were solved via the Simpson numerical integral method:

$$\varphi\sigma \approx \int\limits_{\text{Thermal enegy region}} \varphi(E)\sigma(E)dE \tag{3}$$

By using Eq. (1) and the result of Eq. (3), the activity of ⁶⁴Cu can be calculated and the value was compared with the corresponding experimental value as a benchmark data. In another way, by using the FM4 card the product saturated activity in various energy ranges were calculated.

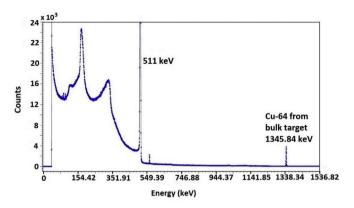


Fig. 3. Gamma spectra of natCu by the gamma HPGe-MCA system.

3. Results and discussion

3.1. Cross section of 63 Cu(n, γ) 64 Cu reaction and theoretical activity value

Fig. 2 shows The cross section of 63 Cu(n, γ) 64 Cu reaction was determined with TALYS-1.8 code and compared with the ENDF/B-VII.1 data [14] and the available experimental data [29]. There are closely connected between them in all energy regions. The calculations by the TALYS code were performed with the resonance parameter. As shown in Fig. 2 at the energy of 0.025 eV the cross section value is 4.47 b. As can be seen in Table 2, the theoretical activity value was calculated about 37.75 GBq by Eq (1) corresponding to the experimental conditions of radiation with cross section of 4.47 b.

3.2. Experimental results

Production yield of ⁶⁴Cu was estimated by its characteristic 1345.84 keV photopeak. The 511 keV photo peak is related to annihilation of ⁶⁴Cu. Figs. 3 and 4 show the spectrums of radioactive products of copper and its nanoscale targets after 80 h cooling time. The counting of gamma emission in 1345.84 keV by HPGe reached to 256 and 85 CPS after 93 min and 18 min for bulk and nanoscale targets respectively. As can be seen in Table 2, the activity value of ⁶⁴Cu at EOB was achieved 37.65 GBg and similarly for its nanoscale target was achieved 42.60 GBq which observed increment activity. The relative difference between two values were calculated 0.116. The ultrahigh surface-to-volume atoms ratio in the nanoscale target were increased and the properties related to the size of copper nanoparticles like cross sections and productions vield can be changed in collision with thermal neutron in nuclear reactor. The specific activity of ⁶⁴Cu can be increased via the long irradiation time and enriched target in less weight. 64Cu is also produced via ⁶⁵Cu(n,2n)⁶⁴Cu reaction with cross section of 280 μb

Table 2 The activity value of 64 Cu at EOB with thermal neutron flux of 4.5×10^{13} n.cm $^{-1}$.s $^{-1}$.

Irradiation time (min)	Theoretical activity (GBq)		MCNPX activity (GBq)		MCNPX	MCNPX activity (GBq)		Experimental activity (GBq)		
	Cross section of 4.47 b	of Relative Difference (with Exp. 1)	FM4-Card (GBq)	Relative Difference (with Exp. 1)	TALYS Code	Relative Difference (with Exp. 1)	Exp. 1 natCu target	Exp. 2 natCuNPs target	Relative Difference (with Exp. 1)	
32	37.75	0.003	38.15	0.013	38.32	0.017	37.65	42.60	0.116	

Relative Difference: (Cal.-Exp.)/Cal.

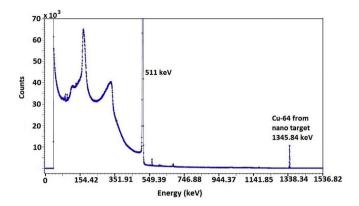


Fig. 4. Gamma spectra of natCuNPs by the gamma HPGe-MCA system.

which was reported by Cohen and et al. [5], however the contribution in total activity is insignificant. ^{66}Cu radionuclide ($T_{1/2}=5.12$ m) was produced with $^{65}\text{Cu}(n,\gamma)^{66}\text{Cu}$ reaction with its characteristic 833 keV (0.22%), 1039.2 keV (9.23%) and 1332.5 keV (0.0037%) photo peaks [30] but with long period cooling time was decayed and there was not in the both spectrums of radioactive samples.

3.3. Simulation results

The neutron energy spectrum is separated in to thermal, epithermal and fast region. The energy range of zero to 0.6 eV is related to thermal region, from 0.6 to 0.5 MeV is related to epithermal region and from 0.5 to 10 MeV is related to fast region too. The thermal region is the activation region for $^{\rm nat}Cu(n,\gamma)^{64}Cu$ reaction so the thermal neutron distribution $\phi(E)$ (n/(cm². s. MeV)) at the $^{\rm nat}Cu$ target was calculated by F4/E4 card-MCNPX. The thermal flux in the target site was calculated about 4.5 \times 10 13 n.cm $^{-2}.s^{-1}$ which is same as the experimental condition. The cross section $\sigma(E)$ (in barn) of $^{\rm nat}Cu(n,\gamma)^{64}Cu$ reaction was calculated via TALYS-1.8 code in thermal neutron energy region and via these results, Eq. (3) was calculated about 2.042 \times 10 $^{-10}$ n/s. At the end, the activity value was calculated by Eq. (1) about 38.32 GBq.

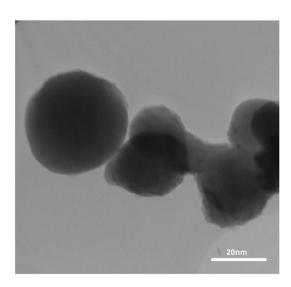


Fig. 5. TEM of copper nanoparticles from Iranian Nanomaterials Pioneers Company, NANOSANY.

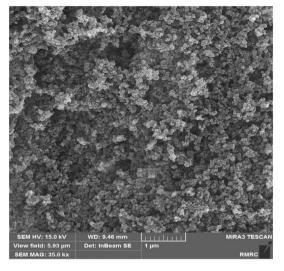


Fig. 6. SEM of copper nanoparticles from Iranian Nanomaterials Pioneers Company (NANOSANY), before irradiation process.

The saturated activity was calculated by FM4 card-MCNPX in three neutron energy regions as can be seen in Fig. 10. The maximum attainable activity was calculated in the thermal neutron energy region about 13.30 \times 10 2 GBq. The activity value after 32 min irradiation time was calculated about 38.15 GB. The activity value of ^{64}Cu predicted by the MCNPX code was listed in Table 2.

3.4. Characterization of natCuNPs

Figs. 5 and 6 show the TEM and SEM images and size distribution graph of natCuNPs respectively. These figures show the size of 20 nm with spherical morphology. Fig. 7 shows the SEM images of natCuNPs after irradiation in reactor core which shows no definite changes compared to nonradioactive samples so the size of 20 nm and spherical morphology of nanoscale copper were remained. Fig. 8 is related to XRD analysis of copper nanoparticles and also Fig. 9 is the XRD analysis after 6 months later when it was placed in the vicinity of the air. The storage condition of nanoscale powder was sealed in vacuum and was stored in cool and dry room and it should not be exposure to air because the damp reunion will affect its dispersion performance. XRD analysis clearly displayed the CuONPs chemical formula with 70 nm crystallite size in pick

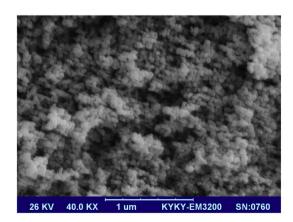


Fig. 7. SEM of copper nanoparticles from Digital Scanning Electron Microscope EM 3200 KYKY, after irradiation process.

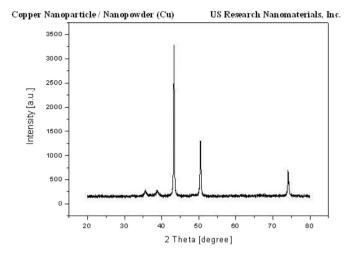


Fig. 8. XRD pattern of the copper nanoparticles from US Research Nanomaterials, Inc.

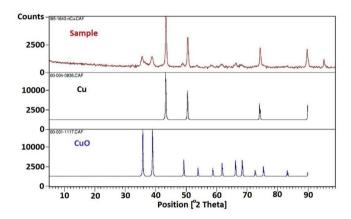


Fig. 9. XRD patterns of copper nanoparticles from STOE STADI, MP, Germany, after 6 months in the vicinity of the air.

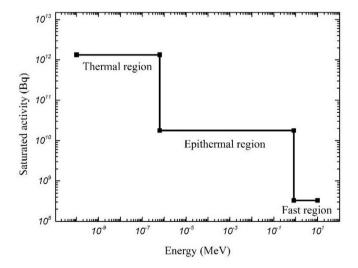


Fig. 10. Saturated activity of 64 Cu via 63 Cu(n, γ) 64 Cu reaction in the different energy regions with FM4 card-MCNPX.

position of 35 were calculated by Eq. (2) and for CuNPs with 41 nm crystallite size in pick position of 43 with the scale factor of 6% and 94% for them respectively.

4. Conclusion

This study, was determined the production yields of ⁶⁴Cu via ⁶³Cu(n,γ)⁶⁴Cu reaction with ^{nat}Cu and ^{nat}CuNPs and showed increment activity of ⁶⁴Cu from ^{nat}CuNPs target. The cross sections values of this reaction were done with TALYS-1.8 code and had good adaptations with theoretical and available experimental cross section data. The activity of ⁶⁴Cu was calculated by using MCNPX code and theoretical methods. The good adaptations were achieved between the experimental results and calculated results, so the MCNPX can be used to optimize production condition in the reactor.

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