



Technical Note

Residual salt separation technique using centrifugal force for pyroprocessing



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ARTICLE INFO

Article history:

Received 23 April 2018

Received in revised form

28 May 2018

Accepted 9 June 2018

Available online 12 June 2018

Keywords:

Pyroprocessing

Molten salt

Salt separation

Liquid-solid separation

Centrifugal force

ABSTRACT

Pyroprocessing uses various molten salts during electrochemical unit processes. Reaction products after the electrochemical processes must contain a significant amount of residual salts to be separated. Vacuum distillation is a common method to separate the residual salts; however, its high operation temperature may cause side reactions. In this study, a simple rotation technique using centrifugal force was suggested to separate the residual salts from the reaction products at relatively low temperature compared to the distillation technique. When a reaction product container with porous wall rotates inside a vessel heated above the melting point of the residual salt, the residual salt in the liquid phase is separated through centrifugal force. It was shown that the $\text{LiNO}_3\text{-Al}_2\text{O}_3$ mixture can be separated by this technique to leave solid Al_2O_3 inside the container, with a separation efficiency of 99.4%.

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1. Introduction

The accumulation of spent nuclear fuels (SNFs) is a global issue, and abatement technology for such high-level radioactive waste should be developed [1,2]. Since it was successfully demonstrated through the EBR-II (Experimental Breeder Reactor-II) project that pyroprocessing can recycle metal fuels of sodium-cooled fast reactors [3], there has been an attempt to employ pyroprocessing for the recycling of SNFs from pressurized water reactors [4,5]. Pyroprocessing is composed of various physical, chemical, and electrochemical unit processes to separate, recycle, and store the radioactive elements in the SNFs [4,5]. Among them, the electrochemical processes (e.g., electrolytic reduction, electrorefining, and electrowinning) use molten salts, such as LiCl and LiCl-KCl (normally, near eutectic composition), as electrolyte materials [4,5]. Therefore, the reaction products of these processes, which were immersed in the molten salts, must contain residual salts with them [6–8]. The residual salts should be properly removed to ensure an adequate quality of the reaction products of each electrochemical process.

Distillation (gas-solid separation) has been commonly accepted for separating the residual salts from the reaction products during pyroprocessing [6–8]. This process requires high temperature (up to 1400 °C) and low pressure (down to vacuum level) to increase the evaporation rate of the salt phases [6–8]. However, a high-temperature operation is not preferred in this system owing to a degradation of the reaction products. For instance, metal products (e.g., metallic U) after the electrolytic reduction can be partially re-converted into the original oxide phase (e.g., UO_2) with the presence of Li_2O in the residual LiCl salt during the high-temperature distillation [8]. This is critical because transferring re-oxidized phases (e.g., UO_2) to subsequent processes results in poor production yield of whole pyroprocessing as the oxide phases cannot be further treated in the electrorefining process. In addition, metallic actinides (dominantly, U) may experience eutectic melting with Fe and other elements in the metal containers (generally, made with stainless steel) of the reaction products at an elevated temperature (e.g., eutectic point of U-Fe system: ~725 °C at 34 at% of Fe) [9]. Although lowering the operation temperature is beneficial to guarantee the quality of the distillation products, it will lead to low distillation efficiency at the same time owing to a reduced vapor pressure of the salt phases, which is the key driving force of this process.

Recently, we proposed a simple liquid-solid separation technique using a heating gun with no vacuum system [10]. Injecting

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heated gas with an adequate flux can melt and remove the salts on the surface of the solid products through the heat and momentum transfer of the high-temperature gas particles. The operation temperature of this technique is maintained near the melting point of the salt phase by controlling the heating gun power and gas flow rate, and thus the above mentioned problems can be avoided. However, it is questionable whether this heating gun technique can be scaled-up for a large-scale pyroprocessing system upon its requirements of significant heat and gas flux.

In this study, we investigated the liquid-solid separation process using centrifugal force to drive the liquid-phase removal from the reaction products. It was expected that the liquid-phase salt can be easily separated from the reaction product when the reaction product container is rotated at an adequate temperature (>melting point).

2. Materials and methods

Figs. 1 and 2 show a schematic design and photographs of the residual salt separation equipment, respectively. A reaction product container (120 mm L × 120 mm H × 20 mm W) is inserted into the outer vessel (φ207 × 232 mm H) to hang the container on the upper part of the vessel. The wall of the container has holes (diameter of 2 mm, equally spaced with distance of 4 mm) to expel the liquid phase from the container so that it can be recovered on the bottom of the outer vessel. The outer vessel is connected to a rotor to obtain a centrifugal force. The rotation speed can be adjusted using a logic-controlled servo-motor (1.5 kW) connected to the rotor through a power transmission. The outer vessel is covered with a furnace for heating to melt the salt phase. The top part of the rotor is water-cooled to protect the servo-motor, bearing, and power transmission from the heat.

Low-melting-point salt, LiNO_3 (255 °C), was chosen as a surrogate material of the salt phases (i.e., LiCl , LiCl-KCl) for experimental convenience. The liquid density of LiNO_3 is approximately 1.75 g cm^{-3} , which is comparable to that of LiCl ($\sim 1.55 \text{ g cm}^{-3}$) and LiCl-KCl ($\sim 1.65 \text{ g cm}^{-3}$). The viscosity of LiNO_3 ($\sim 5.1 \text{ mPa s}$ at 270 °C, $\sim 3.0 \text{ mPa s}$ at 349 °C) is higher than that of LiCl ($\sim 1.4 \text{ mPa s}$ at 647 °C) and LiCl-KCl ($\sim 2.2 \text{ mPa s}$ at 500 °C) [11–13]. Thus, it is considered that the use of LiNO_3 is under harsher conditions than that of LiCl and LiCl-KCl for a liquid-solid separation experiment. Al_2O_3 tubes (Haldenwanger, Germany) of 6 mm in diameter with six longitudinal holes (diameter: 1 mm) were cut into small pellets

(length: 7 mm in average) for the use of a solid material instead of the radioactive SNFs.

The Al_2O_3 pellets were loaded inside the reaction product container, and molten LiNO_3 (99.0%, Samchun Chemical, Republic of Korea) was poured into the Al_2O_3 -loading container to make an Al_2O_3 - LiNO_3 mixture. Then, all of the equipment was assembled for the experiment. The furnace temperature was elevated to 350 °C at a heating rate of 100 °C h^{-1} . After waiting for 30 min at 350 °C, the rotor was rotated at 2000 RPM for 20 min for the separation. Cooling water was circulated during the whole experiment to prevent the heat from over-heating the mechanical parts (servo-motor, bearing, power transmission). For comparison, an additional experiment was conducted without the rotation to clarify the effects of the rotation operation. The waiting time at 350 °C was 50 min in this experiment.

3. Results and discussion

This study was mainly focused on the conceptual separation experiment of the electrolytic reduction products, composed of the metal products (mainly, pelletized form) and residual LiCl (containing Li_2O), because suppressing the re-oxidation behavior is an important issue [8]. The dimension of the Al_2O_3 pellets is set comparable to that of the simulated SNF pellets (diameter: 6.2 mm, length: 7.0 mm), which were developed as feed materials for the engineering-scale electrolytic reduction system [14]. Since the macroscopic structural change of the feed materials before and after the electrolytic reduction is not that remarkable [15], it is thought that use of these Al_2O_3 pellets can simulate the salt separation behavior of the electrolytic reduction products, at least in bulk dimension (pellet-to-pellet and pellet-to-basket). The residual salt removal in the bulk dimension is meaningful for the treatment of the electrolytic reduction products because the most of the residual LiCl is expected to exist in this area. For instance, assuming that the residual salt amount is 20 wt% to the electrolytic products and the porosity of the reduced pellets (metallic U) is 30%, approximately 87% of the residual salt is located in the bulk area and other are inside the pores of the pellets.

Fig. 3 shows the Al_2O_3 pellets and the Al_2O_3 -loading container before and after pouring of the LiNO_3 salt. The loading amount of Al_2O_3 was approximately 254 g (Table 1), which is equivalent to 1228 g of metallic U estimated through a simple comparison of density of both materials (3.95 g cm^{-3} for Al_2O_3 , 19.1 g cm^{-3} for

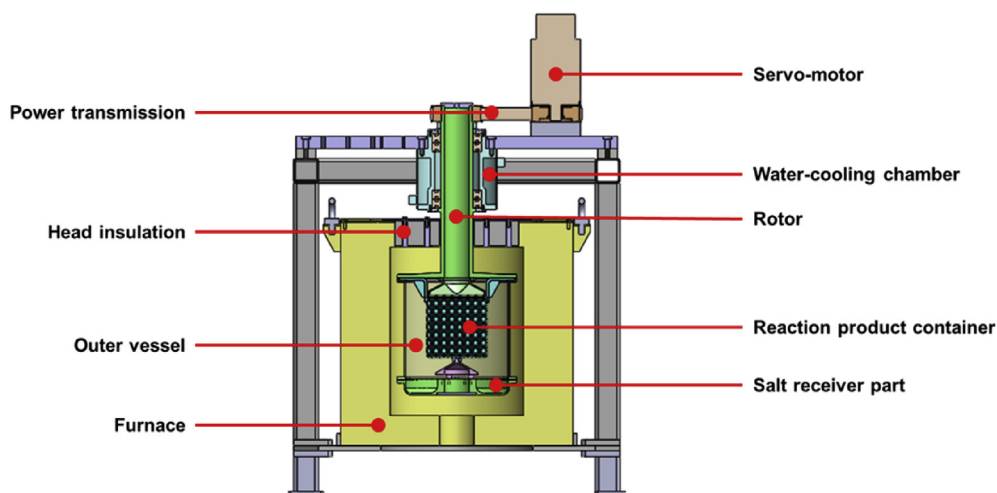


Fig. 1. Schematic illustration of the residual salt separation equipment.

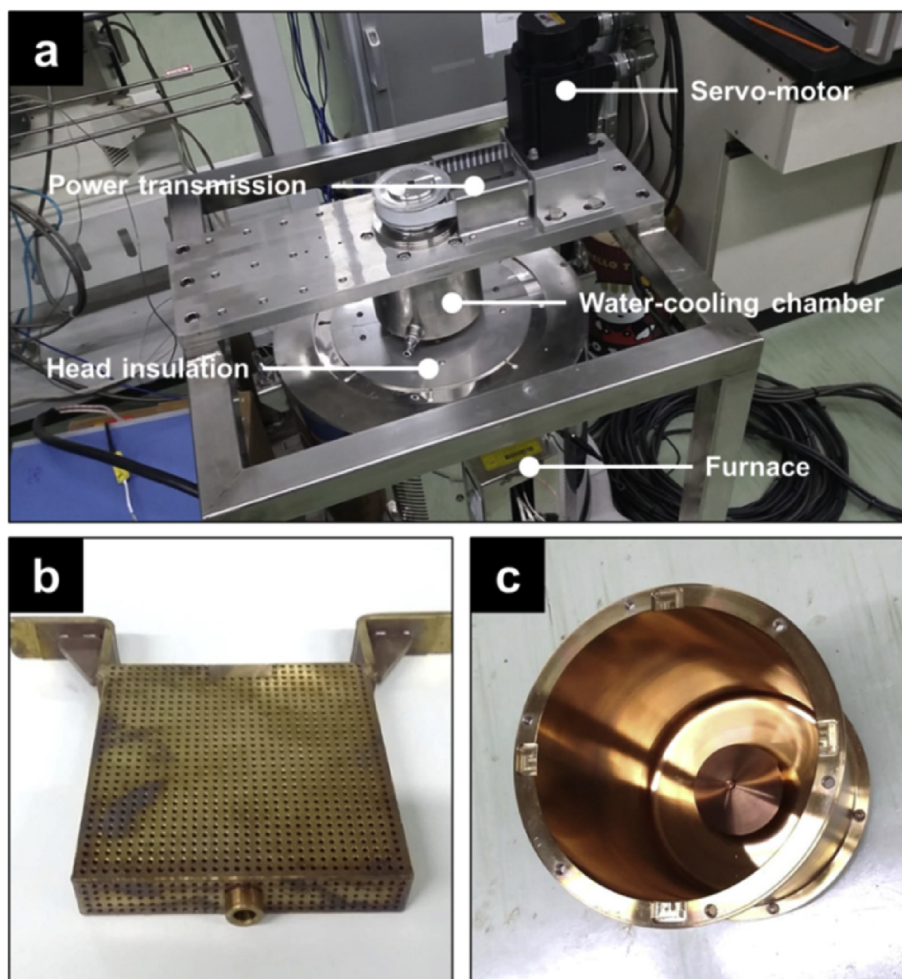


Fig. 2. Photographs of the residual salt separation equipment: (a) assembled structure, (b) reaction product container, and (c) outer vessel.

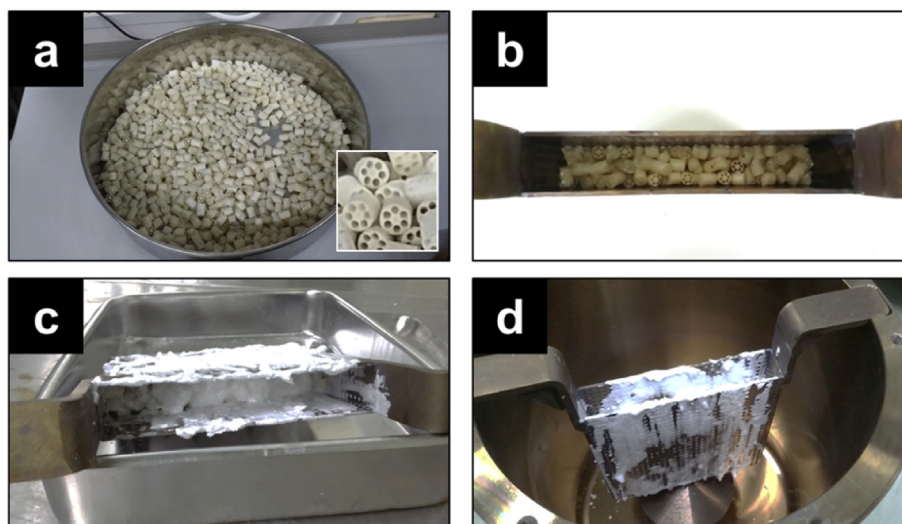


Fig. 3. Photographs of sample preparation procedure: (a) Al_2O_3 pellets, (b) Al_2O_3 -loading container, (c) LiNO_3 -poured container, and (d) LiNO_3 -poured container placed on outer vessel.

metallic U). This scale is consistent to that of the pilot-scale electrolytic reducer under development [16].

After the rotation at 350°C , the LiNO_3 salt was successfully

removed from the container and it was collected at the bottom of the outer vessel, as shown in Fig. 4a and b. The Al_2O_3 pellets were easily recovered by flipping the container upside down. Only a few

Table 1
Summary of residual salt separation experiment with rotation.

Before experiment	Initial Al_2O_3 loading	253.97 g
	Initial LiNO_3 loading	208.28 g
After experiment	Recovered Al_2O_3	254.77 g
	Separated LiNO_3 from container	206.99 g
	Residual LiNO_3 inside Al_2O_3 pellets	0.80 g
	Residual LiNO_3 inside container	0.49 g
	Recovered LiNO_3 inside outer vessel	106.94 g
	LiNO_3 loss	100.05 g

pellets and a negligible amount of LiNO_3 remained inside the container, as shown in Fig. 4c. The remaining pellets inside the container were simply extracted by a small mechanical impact. Fig. 4d shows a magnified image of the recovered Al_2O_3 pellet. The residual LiNO_3 was hardly seen even inside the holes, which indicates that this simple technique is viable for the residual salt separation. Table 1 summarizes the experimental results. The residual amount of LiNO_3 inside the Al_2O_3 pellets and the container was calculated by measuring the weight before and after rinsing them with water. The salt separation efficiency (defined by ratio of separated LiNO_3 from the container to the initial LiNO_3 loading) from the container is approximately 99.4%, demonstrating the superiority of this technique. Only 0.4% and 0.2% of LiNO_3 remained in the Al_2O_3 pellets and the container, respectively. However, only half of the separated LiNO_3 was recovered at the bottom of the outer vessel. It is thought that thermal decomposition of LiNO_3 would greatly affect the salt loss. It was reported that LiNO_3 can be thermally decomposed around the operation temperature evolving O_2 , NO , NO_2 , and N_2 gases [17,18].

It should be noted that gravitation force would be another driving force of the liquid-solid separation. To clarify the effect of the centrifugal force on the separation process, an additional experiment was conducted without rotating the container. No external force except gravity was induced during this experiment. Fig. 5a shows that most of the LiNO_3 on the outer wall and the top of the container was removed well. However, a significant amount of LiNO_3 still remained inside the container as shown in Fig. 5b. Many holes in the container wall were blocked with white-colored material, LiNO_3 . This means that a physical interaction (e.g., surface tension) between the liquid (LiNO_3) and the solid (Al_2O_3 and

container wall) disturbed the liquid phase from moving down. When flipping the container, only a few Al_2O_3 pellets fell down, indicating that most of the Al_2O_3 pellets were strongly combined to the container by the frozen LiNO_3 salt (Fig. 5c). A strong mechanical impact with a hammer was required for the recovery of the Al_2O_3 pellets out of the container. The frozen LiNO_3 crystals were easily seen after the recovery of the Al_2O_3 pellets, as shown in Fig. 5d. Table 2 summarizes the results of this experiment. The separation efficiency is only 76.2%. The overall results indicate that the gravity can lead to liquid-solid separation at a certain level, but additional forces such as centrifugal force must be needed for higher efficiency (>99%).

It was shown that this simple technique is promising for a residual salt separation. For the application of this technique in large-scale pyroprocessing, there are several challenges to be further investigated, as listed below:

- High temperature operation: The melting points of LiCl ($\sim 610^\circ\text{C}$) and LiCl-KCl ($\sim 353^\circ\text{C}$) are higher than that of LiNO_3 and hence a high-temperature operation is required. Under this condition, thermal damage and deformation of the equipment should be investigated. Material stabilities against chemicals such as LiCl and LiCl-KCl also need to be examined.
- Handling the actual reaction products: This study investigated the separation behavior of the residual salts from the reaction products in the pelletized form, to simulate the liquid-solid separation of the electrolytic reduction products. It was demonstrated that this technique is feasible for the residual salt separation at least in the bulk area, where the residual salt exists dominantly. The actual electrolytic reduction products, however, have micro-porous structures in the individual pellets owing to volume reduction during the de-oxidation process [15]. It is not clear at this moment whether the centrifugal force is strong enough to get rid of the residual salts inside the micro-pores. If the residual salts inside the pores are not properly removed, two-step residual salt separation would be considered. Most of the residual salts can be separated during the centrifugal step and the remaining salts can be removed at the subsequent distillation step. This two-step approach can minimize the re-oxidation behavior of the electrolytic reduction products as most of Li_2O , causing the re-oxidation of the metal products [8],

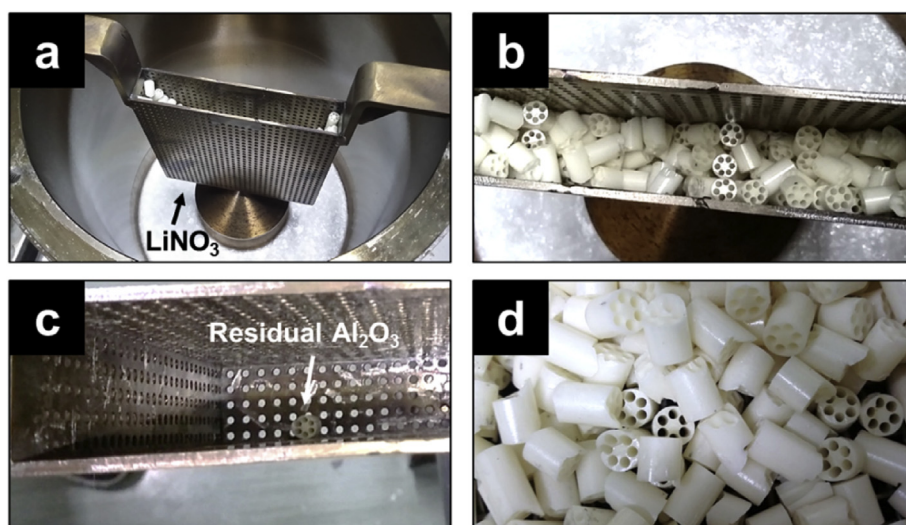


Fig. 4. Photographs of samples after residual salt separation process with rotation: (a) container and outer vessel, (b) Al_2O_3 pellets inside container, (c) inner wall of container after flipping for Al_2O_3 pellet recovery, and (d) recovered Al_2O_3 pellets.

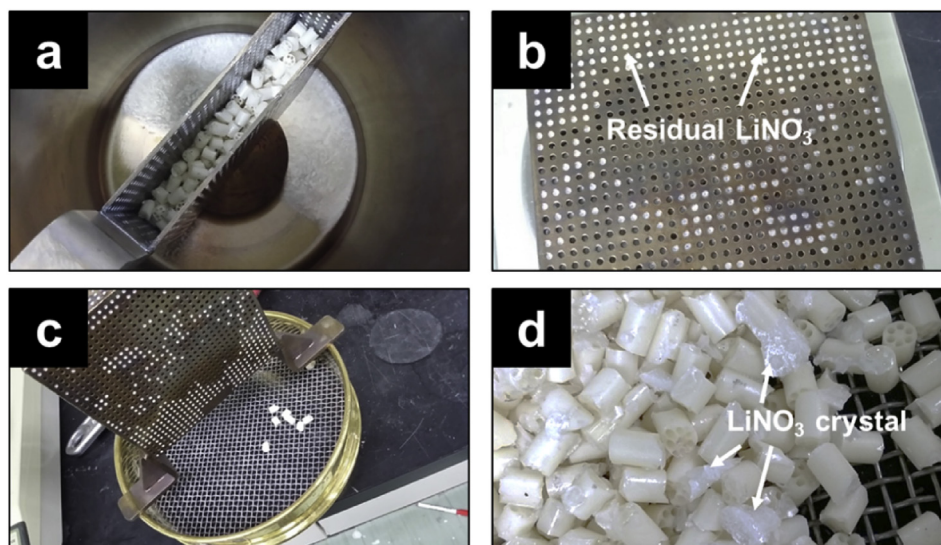


Fig. 5. Photographs of samples after residual salt separation process without rotation: (a) container and outer vessel, (b) outer wall of container, (c) flipped container, and (d) recovered Al_2O_3 pellets with LiNO_3 crystals.

Table 2

Summary of residual salt separation experiment without rotation.

Before experiment	Initial Al_2O_3 loading	253.96 g
	Initial LiNO_3 loading	242.62 g
After experiment	Separated LiNO_3 from container	184.94 g
	Residual LiNO_3 inside container	57.68 g
	Recovered LiNO_3 inside outer vessel	126.30 g
	LiNO_3 loss	58.64 g

can be removed along with the LiCl separation before the high-temperature distillation. Application of this technique to the electrorefining products would be more challenging because the electrorefining products tend to form micro-sized U dendrites with the residual salts [19–21]. The dendrites are deposited on the surface of cathode rods and thus the residual salt separator needs to be designed to handle the whole cathode assembly. Or, the dendrites can be easily recovered by using graphite and W cathodes [19,20], and then the recovered products can be loaded inside the product container for the residual salt separation. Note that the hole size of the container have to be smaller than the size of the reaction products to prevent the escape of the reaction products.

- Design reflecting remote operation procedure: A pyroprocessing facility handles high-level radioactive species, and thus remote operation without human access is required. Various remote operating tools (e.g., manipulator, gantry robot) are under consideration and the design for the process equipment needs to be fit with these tools. In addition, assembly/disassembly operation and maintenance procedures should be simplified for remote operation.

4. Conclusion

In this study, we proposed a simple centrifugal technique to separate the residual salts in the reaction products of the electrochemical unit processes in pyroprocessing. LiNO_3 (liquid) and Al_2O_3 (solid) were chosen as the test materials. The centrifugal force assisted the liquid-phase separation of LiNO_3 from the Al_2O_3 -loading container to achieve a high separation efficiency of ~99.4%.

This technique can be adopted to pyroprocessing; however, further examination should be done on more realistic environments reflecting the operation temperature, test materials, and remote operation procedures.

Conflicts of interest

All authors have no conflicts of interest.

Acknowledgements

This work was supported by the National Research Foundation of Korea (NRF) grant funded by the Korea government (MISP) (2017M2A8A5015077).

References

- [1] R.C. Ewing, Long-term storage of spent nuclear fuels, *Nat. Mater.* 14 (2015) 252–257.
- [2] C. Braun, R. Forrest, Considerations regarding ROK spent nuclear fuel management options, *Nucl. Eng. Technol.* 45 (2013) 427–438.
- [3] C.E. Till, Y.I. Chang, *Plentiful Energy: the Story of the Integral Fast Reactor*, Create Space Independent Publishing Platform, 2011.
- [4] H.-S. Lee, G.-I. Park, K.-H. Kang, J.-M. Hur, J.-G. Kim, D.-H. Ahn, Y.-Z. Cho, E.-H. Kim, Pyroprocessing technology development at KAERI, *Nucl. Eng. Technol.* 43 (2011) 317–328.
- [5] T. Inoue, L. Koch, Development of pyroprocessing and its future direction, *Nucl. Eng. Technol.* 40 (2008) 183–190.
- [6] I.S. Kim, S.C. Oh, H.S. Lim, J.M. Hur, H.S. Lee, Distillation of LiCl from the $\text{LiCl-Li}_2\text{O}$ molten salt of the electrolytic reduction process, *J. Radioanal. Nucl. Chem.* 295 (2013) 1413–1417.
- [7] J.-H. Lee, Y.-H. Kang, S.-C. Hwang, J.-B. Shim, B.-G. Ahn, E.-H. Kim, S.-W. Park, Electrodeposition characteristics of uranium in molten LiCl-KCl eutectic and its salt distillation behavior, *J. Nucl. Sci. Technol.* 43 (2006) 263–269.
- [8] E.-Y. Choi, M.K. Jeon, J.-M. Hur, Reoxidation of uranium in electrochemically reduced simulated oxide fuel during residual salt distillation, *J. Radioanal. Nucl. Chem.* 314 (2017) 207–213.
- [9] C.P. Wang, Z.S. Li, W. Fang, X.J. Liu, Thermodynamic database and the phase diagrams of the (U, Th, Pu)-X binary system, *J. Phase Equilibria Diffusion* 30 (2009) 535–552.
- [10] S.-W. Kim, J.-M. Hur, D.H. Heo, S.-S. Hong, E.-Y. Choi, Residual salt separation of reaction products in pyroprocessing, in: *Korean Radioactive Waste Society 2017 Fall Meeting*, Oct. 18–20, 2017, Changwon, Republic of Korea.
- [11] G.J. Janz, S.W. Lurie, G.L. Gardner, Viscosity of molten LiNO_3 , *J. Chem. Eng. Data* 23 (1978) 14–16.
- [12] M. Wakao, K. Minami, A. Nagashima, Viscosity measurements of molten LiCl in the temperature range 886–1275K, *Int. J. Thermophys.* 12 (1991) 223–230.
- [13] J.-Y. Kim, S.-E. Bae, D.-H. Kim, Y.S. Choi, J.-W. Yeon, K. Song, High-temperature

- viscosity measurement of LiCl-KCl molten salts comprising actinides and lanthanides, *Bull. Kor. Chem. Soc.* 33 (2012) 3871–3874.
- [14] S.-C. Jeon, J.-W. Lee, J.-Y. Yoon, Y.-Z. Cho, Scaling up fabrication of UO₂ porous pellets with a simulated fuel composition, *J. Nucl. Fuel Cycle Waste Technol.* 15 (2017) 343–353.
- [15] E.-Y. Choi, J.-K. Kim, H.-S. Im, I.-K. Choi, S.-H. Na, J.W. Lee, S.M. Jeong, J.-M. Hur, Effect of the UO₂ form on the electrochemical reduction rate in a LiCl-Li₂O molten salt, *J. Nucl. Mater.* 437 (2013) 178–187.
- [16] S.-W. Kim, D.H. Heo, S.K. Lee, M.K. Jeon, W. Park, J.-M. Hur, S.-S. Hong, S.-C. Oh, E.-Y. Choi, A preliminary study of pilot-scale electrolytic reduction of UO₂ using a graphite anode, *Nucl. Eng. Technol.* 49 (2017) 1451–1456.
- [17] M.G. Adamson, D. Calef, R.W. Moir, Lithium nitrate as a fusion reactor coolant fluid?: a thermochemical assessment, *J. Fusion Energy* 5 (1986) 247–252.
- [18] C.M. Kramer, Z.A. Munir, J.V. Volponi, Simultaneous dynamic thermogravimetry and mass spectrometry of the evaporation of alkali metal nitrates and nitrides, *J. Therm. Anal.* 27 (1983) 401–408.
- [19] C.H. Lee, T.-J. Kim, S. Park, S.-J. Lee, S.-W. Paek, D.-H. Ahn, S.-K. Cho, Effect of cathode material on the electrorefining of U in LiCl-KCl molten salts, *J. Nucl. Mater.* 488 (2017) 210–214.
- [20] Y.H. Kang, J.H. Lee, S.C. Hwang, J.B. Shim, E.H. Kim, S.W. Park, Electrodeposition characteristics of uranium by using a graphite cathode, *Carbon* 44 (2006) 3113–3148.
- [21] T. Koyama, M. Iizuka, Y. Shoji, R. Fujita, H. Tanaka, T. Kobayashi, M. Tokiwai, An experimental study of molten salt electrorefining of uranium using solid iron cathode and liquid cadmium cathode for development of pyrometallurgical reprocessing, *J. Nucl. Sci. Technol.* 34 (1997) 384–393.