

Estimation of Burnup with Caesium isotopes based on Gamma-scanning of a instrumented fuel capsule(02F-11K) in Hot-cell

Ungsup Song, Heemoon Kim, Daegy Park, Seungje Paik, Honggi Lee, Yongsun Choo, Kwonpyo Hong
Irradiated Materials Examination Facility, Korea Atomic Energy Research Institute, Dujin, Yusong, Taejeon,
Korea, 305-353 (nussong@kaeri.re.kr)

1. Introduction

Many experimental inspections have been performed to obtain the burnup of fuel. In the case, chemical analysis was popular with high reliability. High radioactivity of fuel was a severe problem during destructive procedure. Afterward, many researchers have studied calculation of burnup using gamma detectors as the non-destructive method^[vi]. Methodologies of gamma-scanning tests have been developed as well as higher accuracy of detector.

Generally, Cs-137 and Cs-134 are standard isotopes for long-term cooling spent fuel to estimate burnup, because atomic ratio of them follows the linearity with burnup.

2. Experimental

2.1 Specimen preparation

Five UO₂ pellets were made with 2.42% of enrichment and contained a small fuel rig which was placed in an instrumented capsule for irradiation^[vii]. Two and half of pellets from the top position were punctured at the center of them to set up a thermocouple. The capsule was irradiated for 55 days with 300 w/cm ~ 330 w/cm of linear power in the OR hole in the HANARO research reactor. After irradiation and cooling, it was moved to a Hot-cell to do gamma-scanning tests.

2.2 Gamma-scanning test

The fuel rig contained irradiated five UO₂ pellets set vertically to the bench device. It can be moved up and down. Two punctured pellets, half punctured one and two solid ones were placed in order from top of fuel rig. Three points for gamma-scanning tests were selected as top, middle, and bottom of the rig as shown in Fig. 1.

The distance between pellet and detector is 1.6 m including thickness of Hot-cell wall. The slit size of Collimator is 1 mm x 1 mm as square type. First of all, background checking was performed before the experiment for the fuel rig to consider special peaks from other materials which make severe counting errors. After gamma-scanning test for fuel rig, that for Cs-137 standard source, known activity, was done with same way. Detecting time was set up by 10⁶ sec.

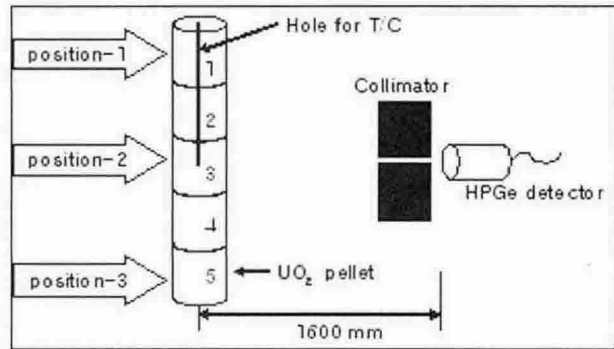


Fig. 1 structural geometry for gamma-scanning test

3. Results and Discussion

Atomic ratio of Cs-137 to Cs-134 is available to calculate burnup because this ratio is linearly proportional to burnup. Using ORIGEN-2 code^[viii], burnup related to this ratio can be found. To obtain the ratio (R), the equation is as follows^[ix]:

$$\frac{N_{134} \epsilon(E_{137})}{N_{137} \epsilon(E_{137})} = \frac{N_{134}}{N_{137}} = R_{\text{exp}} \quad (1)$$

Where, N is atomic amount and ϵ is energy efficiency of detector, energy in here is 662 keV (gamma energy of Cs-137). R is experimental value.

Efficiency (ϵ) at 662 keV was obtained from interpolation with several energy peaks of Cs-134 with equation as follows;

$$C(E_i) = \lambda N P(E_i) \epsilon(E_i)$$

Where, C is gamma counts, λ is decay constant, N is atomic amount, P is decay branch ratio with energy. λN is radioactivity in here.

But, every seven gamma energies of Cs-134 are not shown in high peaks. In this study, just two gamma energies were available due to very small peaks of the others. High peaks of 605 keV and 796 keV were chosen to calculate efficiency at 662 keV.

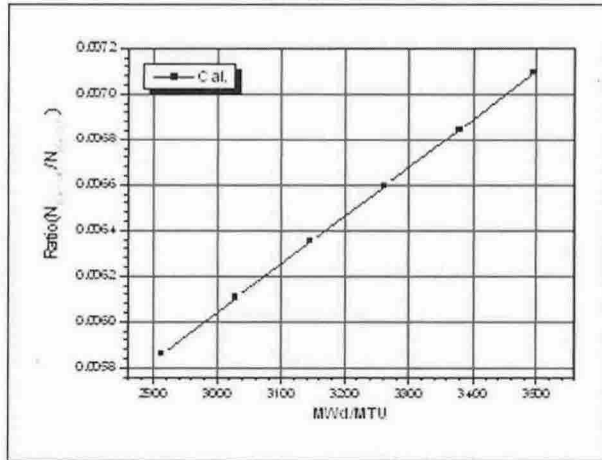


Fig. 2 plot of ratio Cs-134 to Cs-137 with burnup (ORINGEN-2)

R-values from measurement were 0.00658 at top position(no. of pellet is 1), 0.00659 at middle position(no. of pellet is 3) and 0.00664 at bottom position(no. of pellet is 5), respectively. Burnup is approximately 3.22~3.26 Gwd/MTU. Linear power of this fuel rig is assumed to be about 275~285 w/cm considering irradiation time and pellet weight, but this data would be under-estimated by ~10% comparing the calculation neutronics code for HANARO.

4. Conclusion

5 UO₂ pellets were irradiated in reactor for 55days and moved to Hot-cell. Gamma-scanning test was performed at three point of fuel rig contained pellets. Ratios of Cs-134 to Cs-137 were measured in each points. Burnups related to measured ratios were found using ORIGEN-2 code. The results were approximately 3.22~3.26 Gwd/MTU. It is assumed to be under-estimated comparing the calculation of neutronics code for HANARO.

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